

Negative Hydrogen Ion Sources for Fusion Tutorial for NIBS 2020

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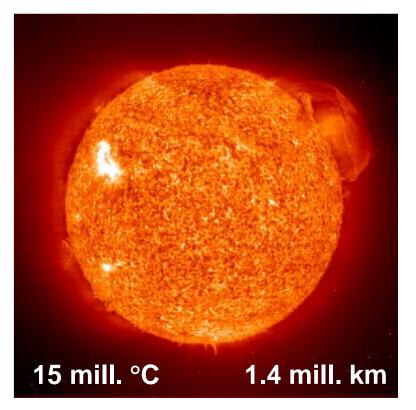
on behalf of the ITER NBI contributors



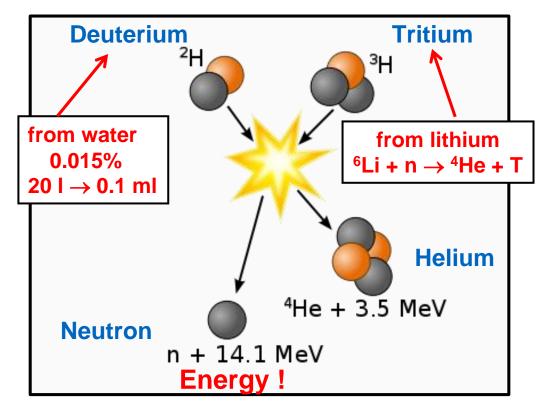
Fusion – The energy source of the sun



Hydrogen ⇒ Helium



on Earth: Hydrogen isotopes ⇒ Helium



4H → ... → ⁴He + ... + 26.7 MeV β^+ decay $D + T \rightarrow {}^{4}He + n + 17.6 \text{ MeV}$

Fusion on Earth needs 10 times higher temperature as in the sun!

The fusion experiment ITER





Largest multinational scientific mission.

1985: Project starts 2006: ITER Agreement officially signed 2019: > 65% ready

To demonstrate the scientific and technological feasibility of fusion power for peaceful purposes.

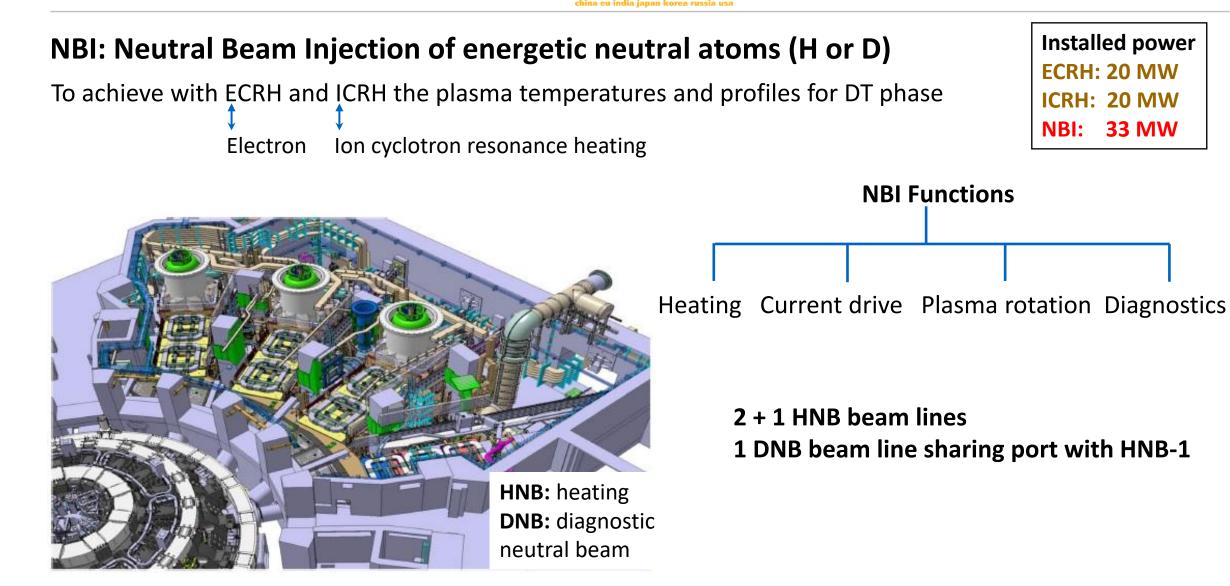
To produce a burning plasma.

Q>10 for 400 s (Q > 5 for 3600 s) Output (fusion power): 500 MW Input (heating power): 50 MW

Size: 24 m diameter, 30 m height Weight: 23 000 tons (3 x Eiffel tower)

Neutral beam systems for ITER



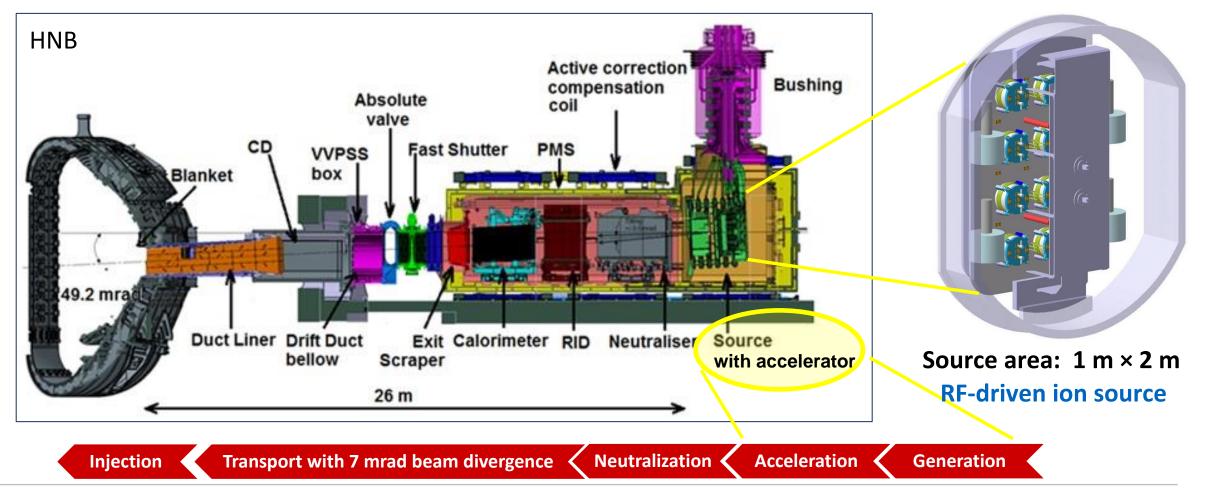


ITER NBI systems and their requirements





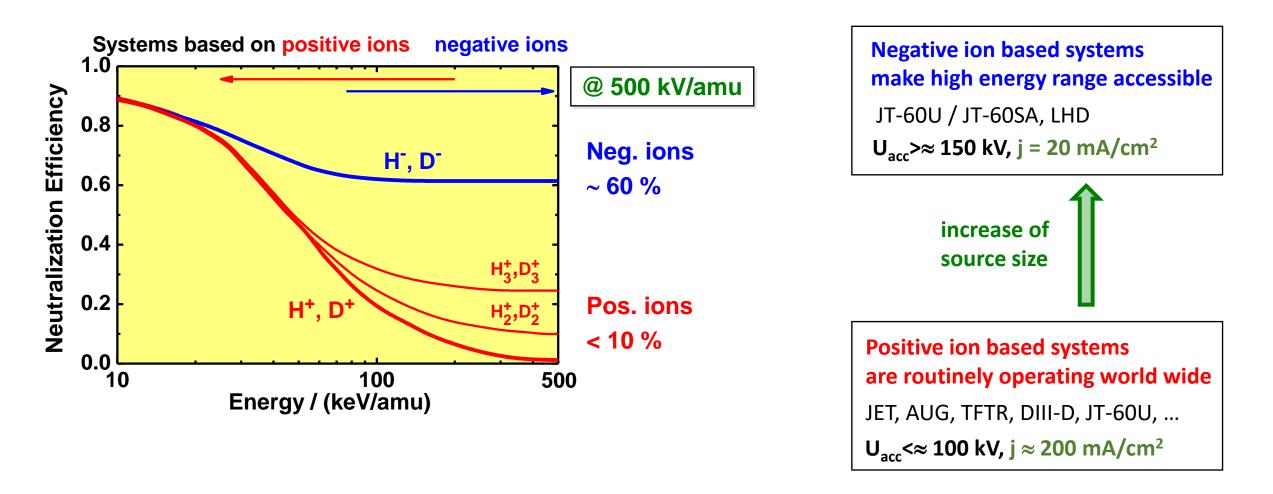
Heating beams (50% EU, 50% JA) : **33 MW** (2 injectors) for 3600 s, **1 MeV Deuterium, 870 keV Hydrogen** Diagnostic beam (100% IN): **2.2 MW, 100 keV Hydrogen**, 3s ON/20s OFF 5Hz



Why negative hydrogen ions?



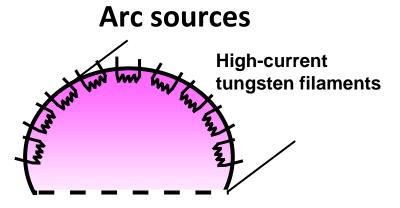
Neutralisation efficiency at a beam energy of 1 MeV D



Concept of ion sources – Arc sources and RF-driven sources

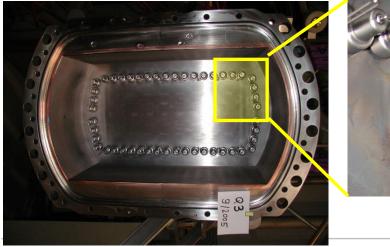
RF

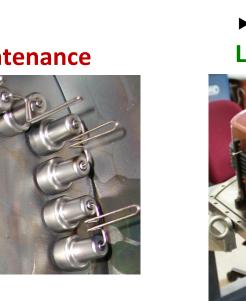


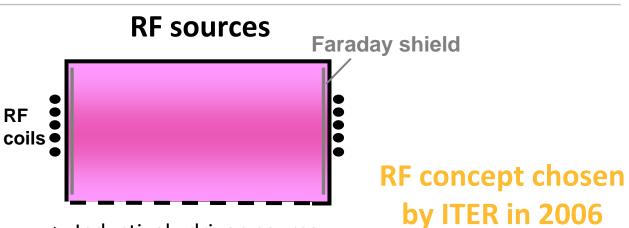


- ▶ Hot cathodes (2000 3000 K)
- DC voltage (≈ 100 V)
- Arc current (1000 A)

Filaments require regular maintenance

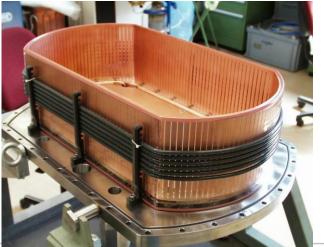


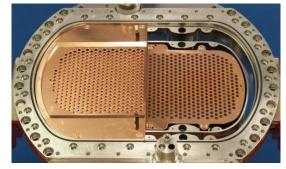




- Inductively driven source
- ▶ RF power supply (\approx 100 kW)
- ► RF frequency 1 MHz

Long lifetime, routine operation for positive ions at AUG

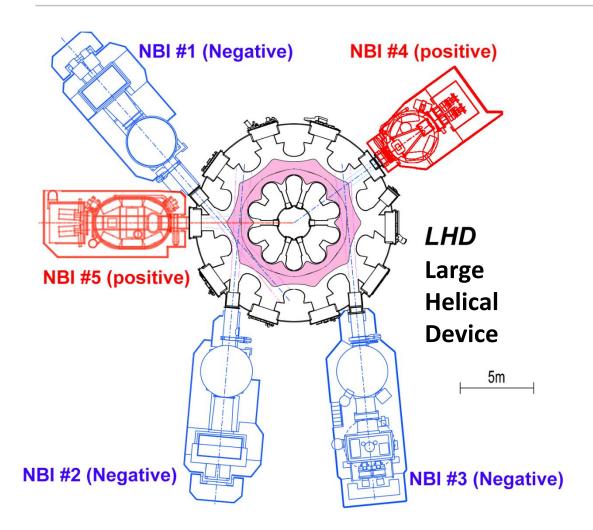




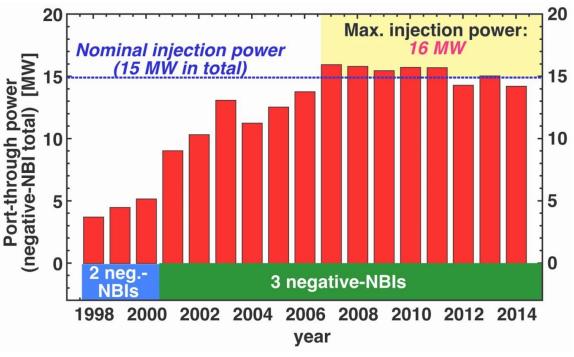
Multi-aperture grid system (AUG) 774 apertures, 8 mm in diameter **3** grids for acceleration & focussing

NBI systems at LHD at NIFS, Japan





	negative	positive
Beam energy [keV]	190	80 & 90
Injection power [MW]	5.5 - 6.9	9
Pulse length [sec]	10 (max)	10 (max)
Beam divergence [mrad]	5	11

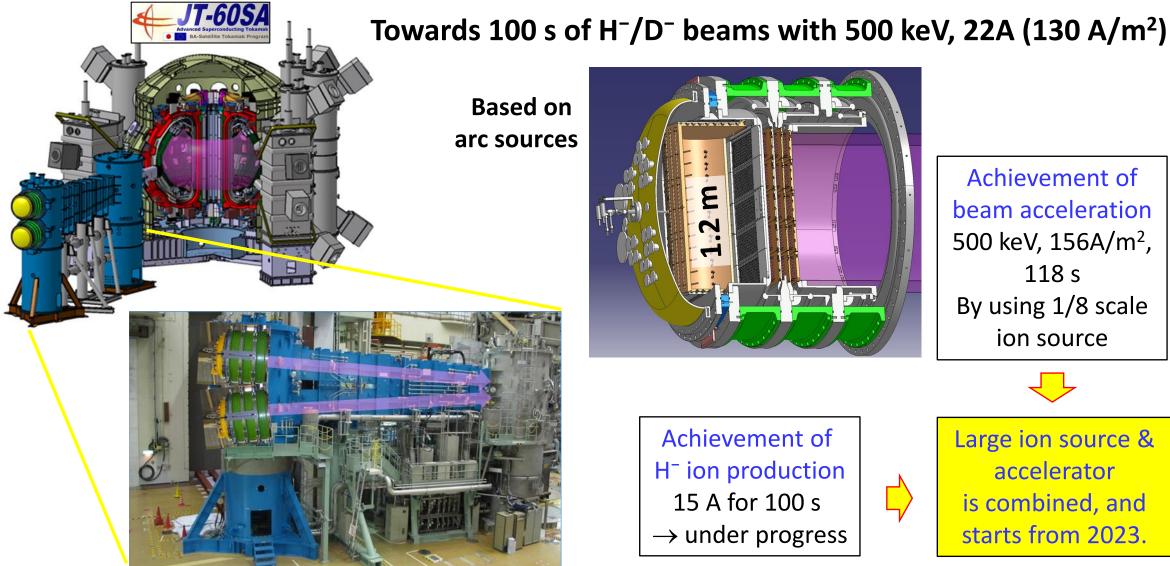


Arc sources, operation mostly in hydrogen

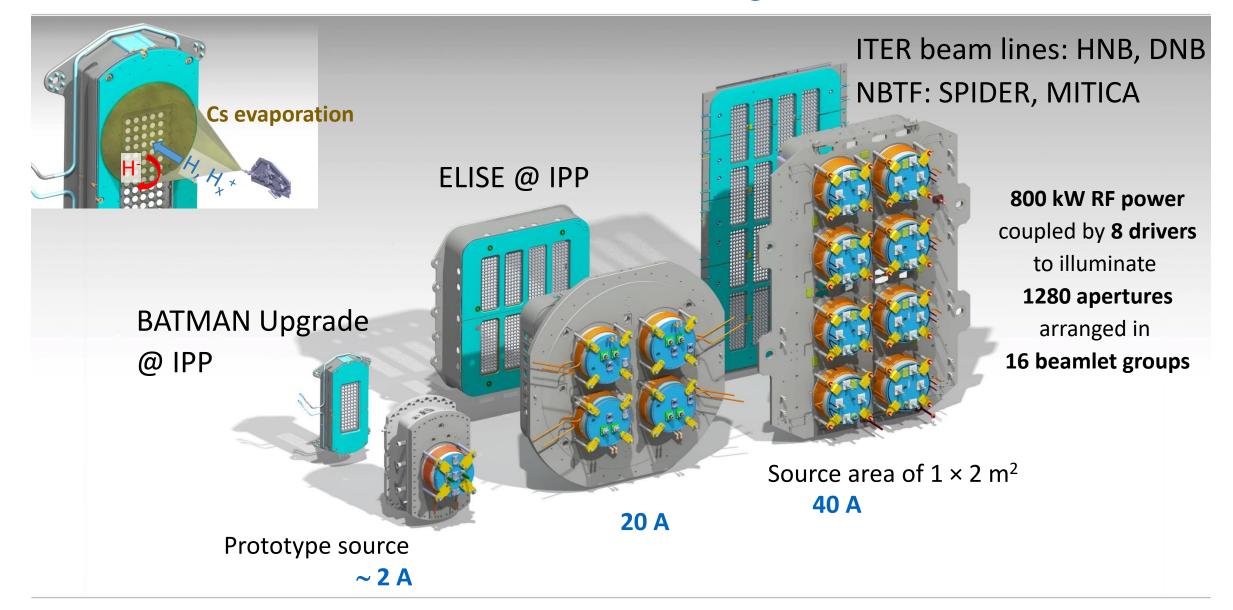
K. Tsumori, Fusion Sci. Tech. 58, (2011) pp.489

NNBI systems at JT-60U / JT-60SA at QST, Japan





R&D for the ITER ion source – a size scaling route



The test facility for

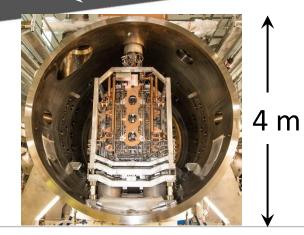
SPIDER @ 100 keV

started in June 2018



NBTF **Neutral Beam Test Facility**

Full ITER beam line



Critical challenges:

MITICA @ 1 MeV

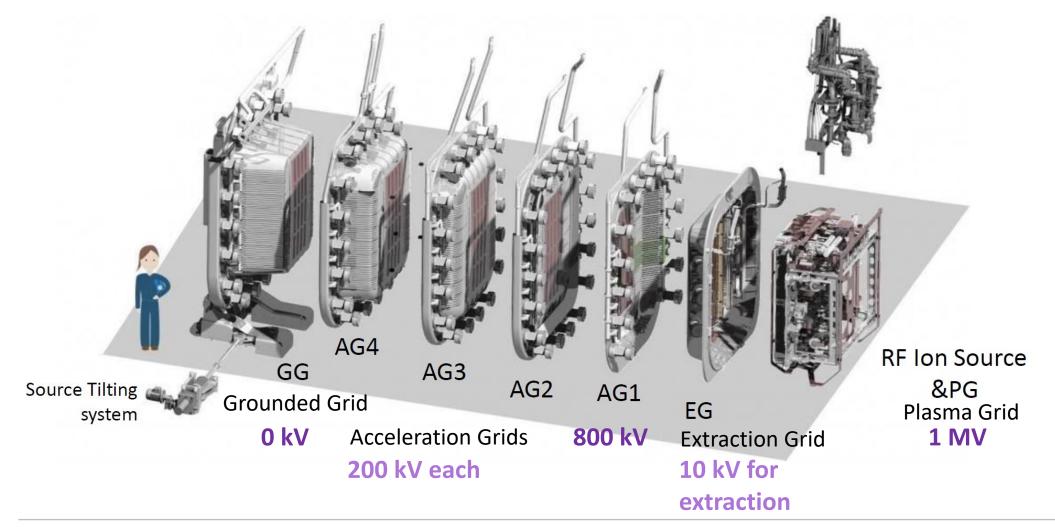
starts in 2023

TTTTTT

- Extraction of 40 A negative ion beam from a large-size RF source
- Acceleration 1 MeV with accurate beam optics
- Development of high-voltage, gas-insulated transmission lines
- Voltage holding (1 MV) over pulses of 3600 seconds



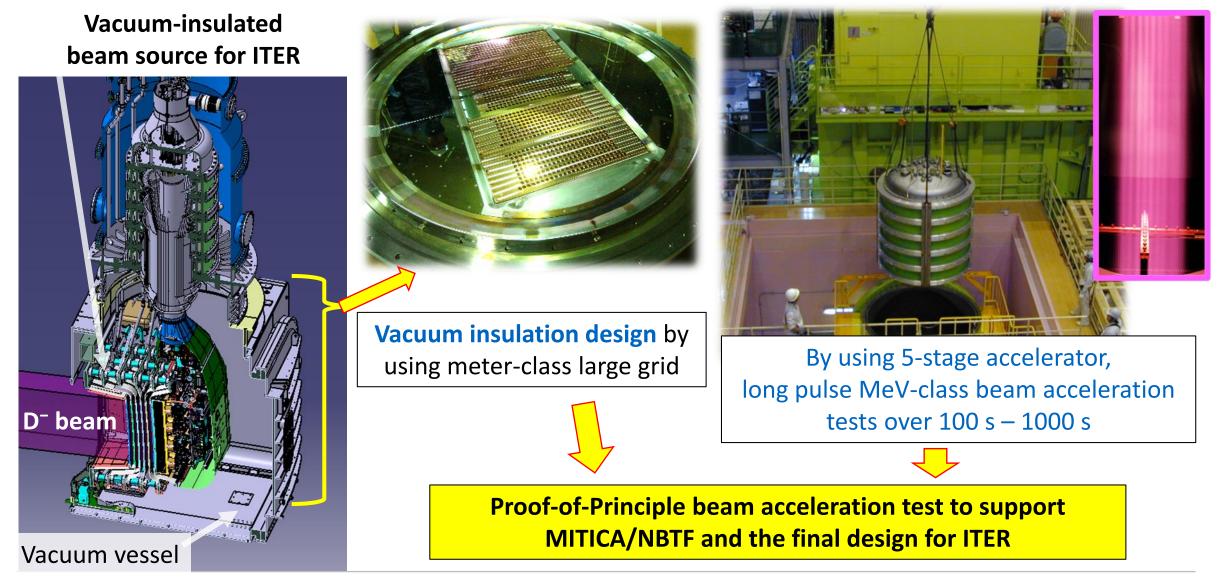
The beam source of MITICA (full size HNB prototype)



The 1 MeV acceleration R&D at QST, Japan







September 1, 2020

ELISE – A half size ITER source

First plasma and beam: Feb. / Mar. 2013

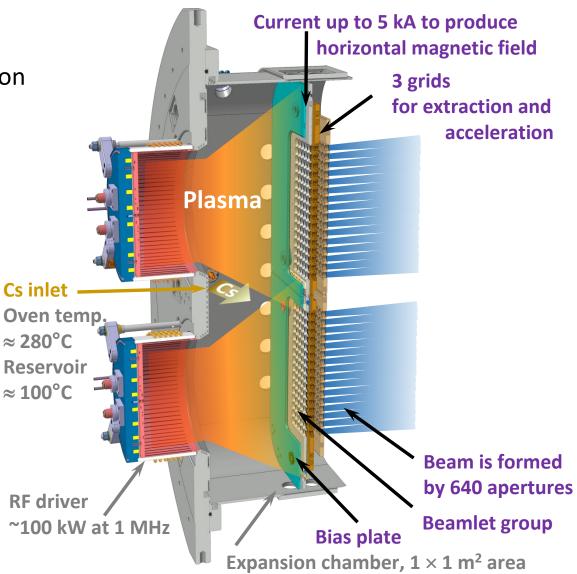


ELISE is dedicated to

- Provide input for design, commissioning and operation of ITER NBI systems
- ► **Demonstrate** ITER parameters in large sources
 - Extracted currents (ions and electrons)
 - Beam homogeneity
- Develop most efficient source operation scenarios

Parameter and targets

Isotope D ⁻ (H ⁻)	
RF power = 2 x 150 kW in 4 drivers	
A _{ex} = 1000 cm ² , uniformity > 90%	
$I_{ion,ex} = 29 (33) A, I_e/I_{ion} < 1 at 0.3 Pa$	
U_{tot} = 60 kV, U_{ex} < 12 kV	
Plasma: 3600 s	
Beam: 10 s every ~150 s (HV supply)	



ELISE – A half size ITER source

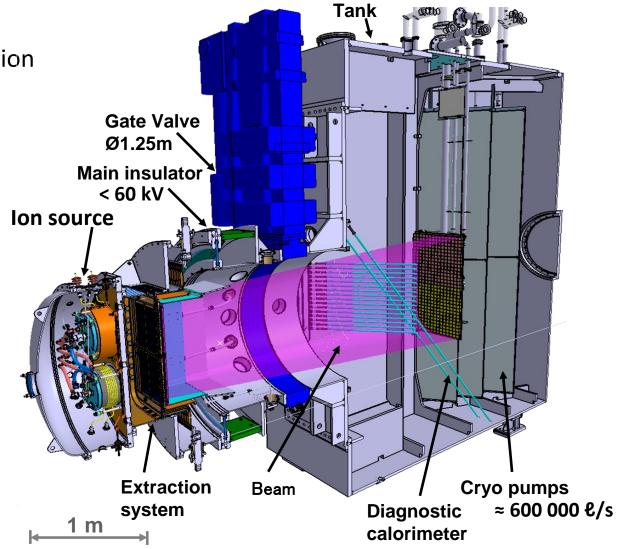


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Parameter and targets

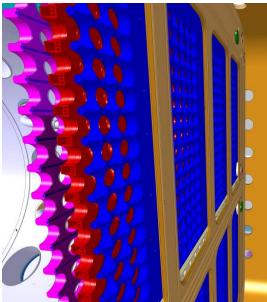
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The grid system

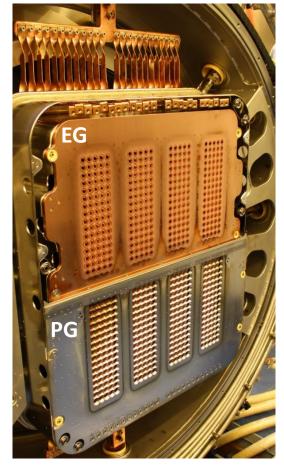


Three grids



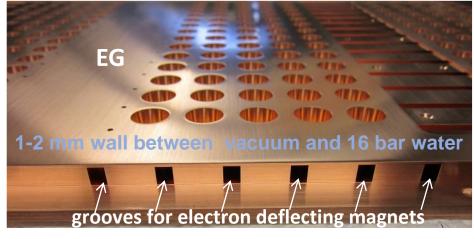
GG EG PG BP Plasma Extraction Grounded Grid

Two segments

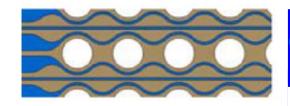


PG and inner surfaces of the ion source are coated with molybdenum to avoid Cs interaction with Cu.

Extraction grid



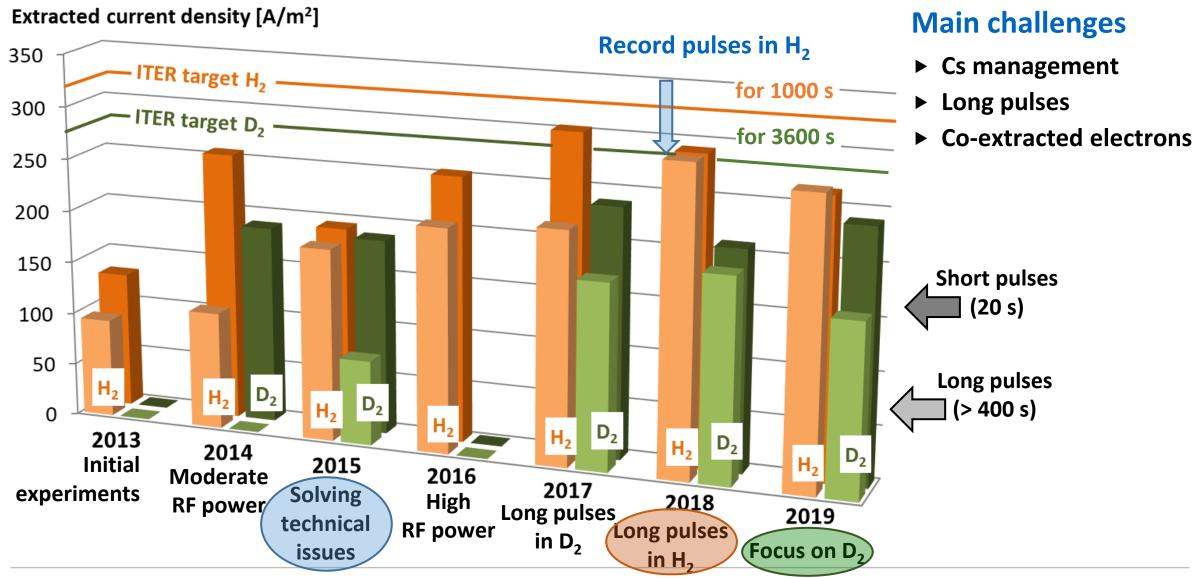
altering direction from row to row



Power density of 32 MW/m² by co-extracted electrons

Present status – Source performance





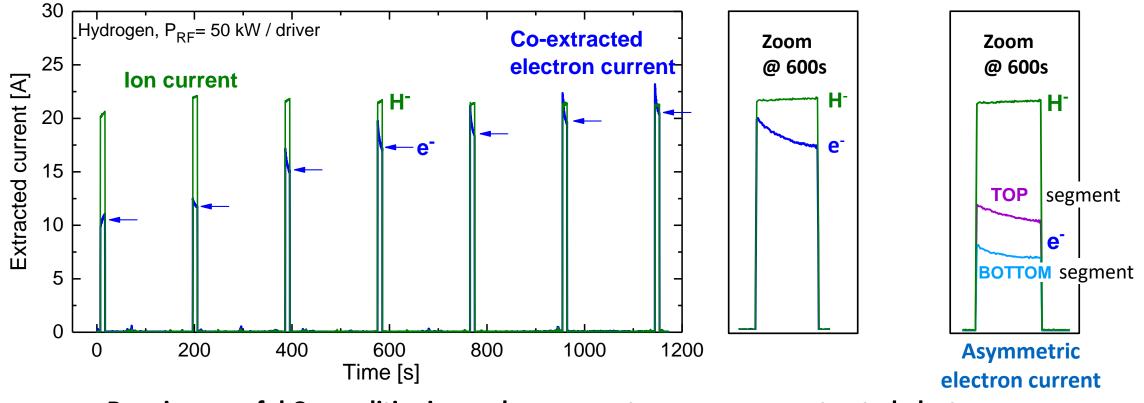
Typical long pulse behaviour



Source performance is probed by short pulse extraction of 10 s every 3 min

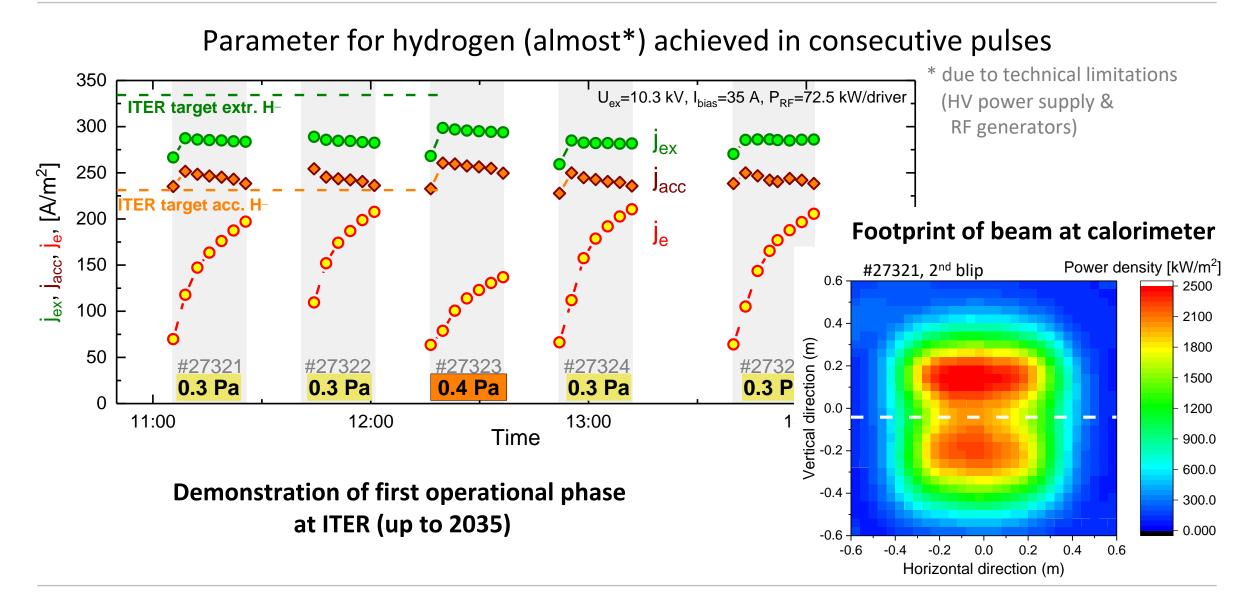
- **Stable** negative ion current density (within 10%)
- Strong temporal dynamics of co-extracted electrons

Interlock of extraction grid is set to 125 kW/segment although designed for 200 kW (ITER: 600 kW for all 4 segments)



Requires careful Cs conditioning and measures to suppress co-extracted electrons

Achievements of ELISE in 2018 – Towards ITER targets



Deuterium operation – Achievement so far: 200 A/m² (67%) for almost 1 h

Strong isotope effect in terms of co-extracted electrons

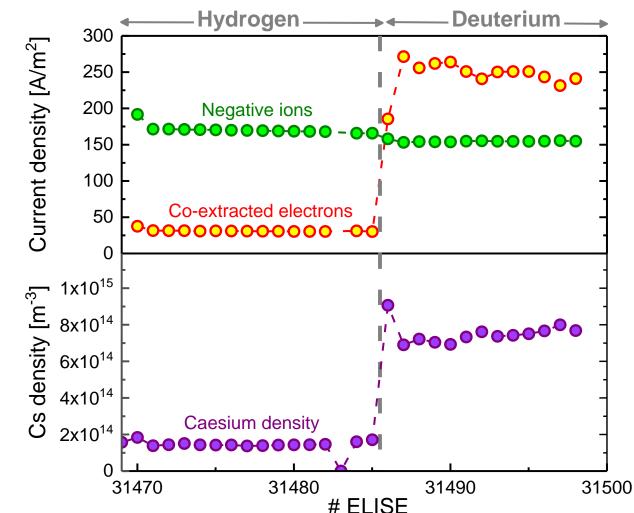
Transition from hydrogen to deuterium at identical source parameters

- Drastic increase of co-extracted electrons
- Strong increase of Cs density close to plasma grid

at almost the same ion current density

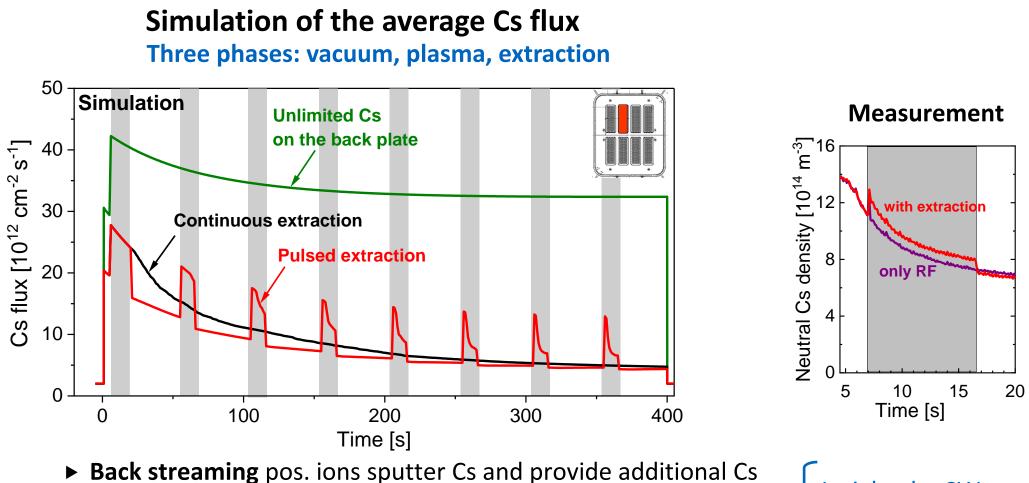
In general: co-extracted electrons

- ▶ are factor 2 4 higher in D
- Imit the source performance
- ▶ require more Cs (~ factor 2)



One of the key elements – The Cs dynamics





- ► Continuous extraction ⇒ still not sufficient to stabilize Cs flux
- ► Unlimited Cs reservoirs in the back-plate: higher and stable flux

Insights by CW extraction at SPIDER (soon at ELISE)



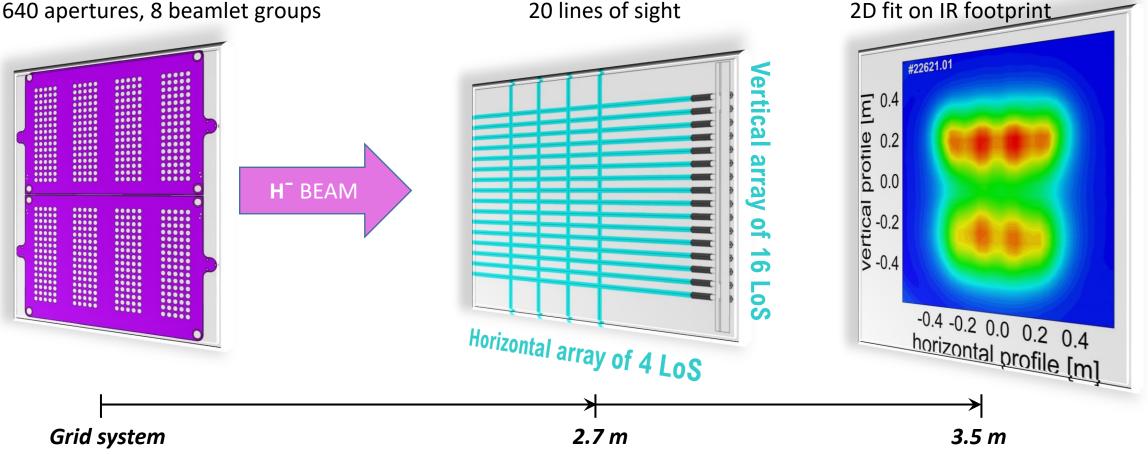
IR calorimetry

Diagnostics for beam divergence and homogeneity

Beam emission spectroscopy

Arrangements of apertures

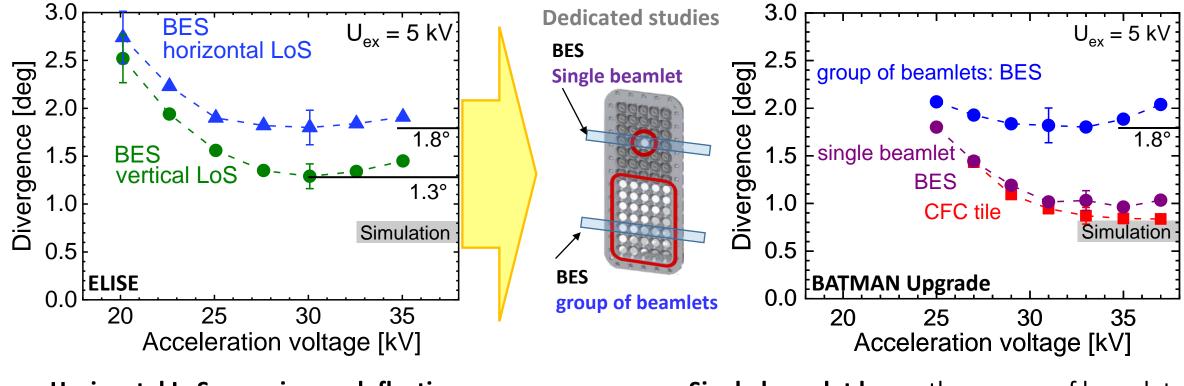
640 apertures, 8 beamlet groups



Beam characterisation: group of beamlets - single beamlet

ITER requires a divergence of < 7 mrad (0.4°) in the core of a single beamlet

ELISE grid system: simulation give a divergence of 0.6 -0.8° (3 grids with max. 60 kV acceleration)

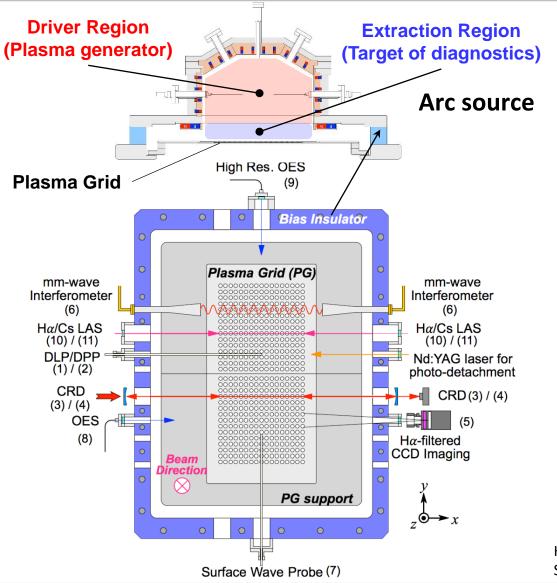


Horizontal LoS sees zig-zag deflection caused by deflection field (EG magnets) Single beamlet lower than group of beamlets Agreement with simulation

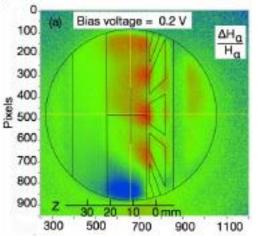
ITER R&D at dedicated test facility at NIFS, Japan





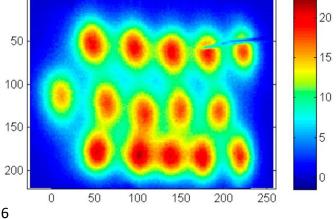


Versatile diagnostics of plasma and beam for fundamental understanding



 ${\rm H}_{\alpha}$ image of ${\rm H}^{-}$ extracted distribution

5 x 3 beamlet pattern on a CFC tile monitored with infrared camera



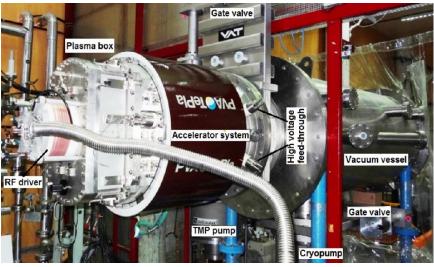
K. Ikeda *et. al., New J. Phys.* **15** (2013) 103026 S. Geng *et. al., Fusion Eng. Des.* **121** (2017) 481

ITER R&D at IPR, India



Roadmap : Beam (operational experience) and technology development in parallel Learning curve on 3 test beds : ROBIN, TWIN, INTF

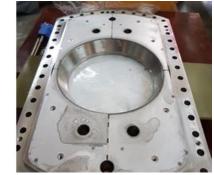
ROBIN TEST BED



- ► 27 mA/cm² H⁻ beams @ 25 keV
- High Cs consumption (impurity control)
- ▶ e⁻/H⁻ > 1
- Experiments restarted after cesiated source cleaning



before and after Cs cleaning



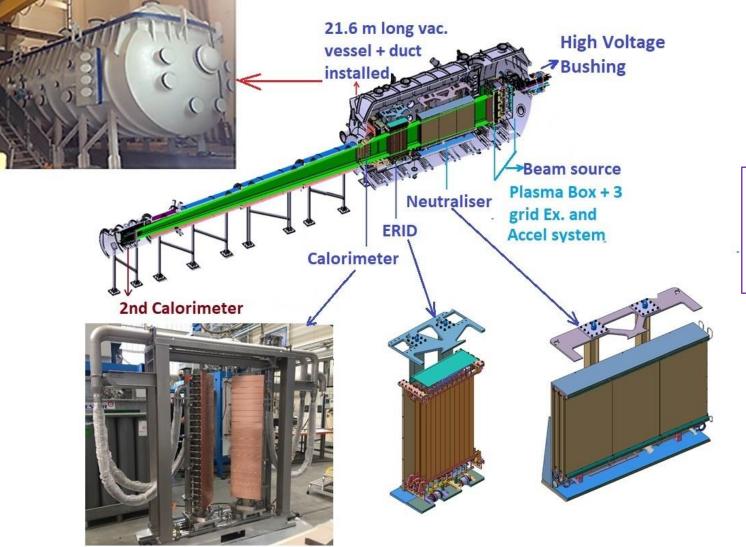
TWIN TEST BED



- Plasma production exp. initiated (50 kW two drivers)
- ► RF generator problems
- Accelerator system under proc.

ITER diagnostic beam developed at IPR, India





INTF @ ITER-India lab Protoype DNB beam line Unique 21.6 m path length to establish beam parameters and transport

- Several technologies developed enroute
- Components (DNB) under fabrication

Integration and commissioning : Q3 2021



Ion sources for fusion – Take-Home message

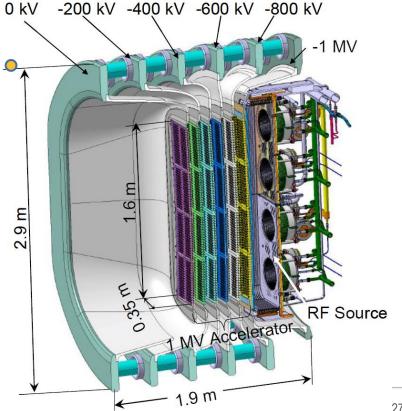
- Strong activities for ITER to make it a success!
- International coordination, only feasible with high commitment of participating institutes to ITER.
- Cutting edge physics and technology.
- We are on a good path with many contributions with distributed responsibilities and know-how.
- ITER is prepared with the NBI R&D activities worldwide. In fact, NBTF is the first ITER facility in operation.

Still huge challenges in front of us

- Achievement of Deuterium target values
- Co-extracted electrons limiting the source performance
- Cs management for large sources
- 1 MeV holding and beam acceleration with accurate optics
- Reproducibility and reliability

Fact Sheet

40 A, 1 MeV D⁻ for 1 h 46 A, 0.87 MeV H⁻ 60 A, 100 keV H⁻ for DNB 800 kW RF (8 drivers), 0.3 Pa 7 Electrodes 15 beamlet groups 1280 beamlets -200 kV -400 kV -600 kV -800 kV





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